

# Accuracy and Runtime Improvements with SCALE 6.2

Presented to:

## SCALE Users' Group Workshop

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# SCALE 6.2 – April 2016

Innovative

- Modernized architecture for efficiency and quality
- Enhanced sensitivity and uncertainty analysis
- Problem-dependent temperature treatments for continuous-energy Monte Carlo
- Reference continuous-energy depletion

Efficient

- Accelerated lattice physics capabilities
- Reduced memory requirements
- Parallel calculations

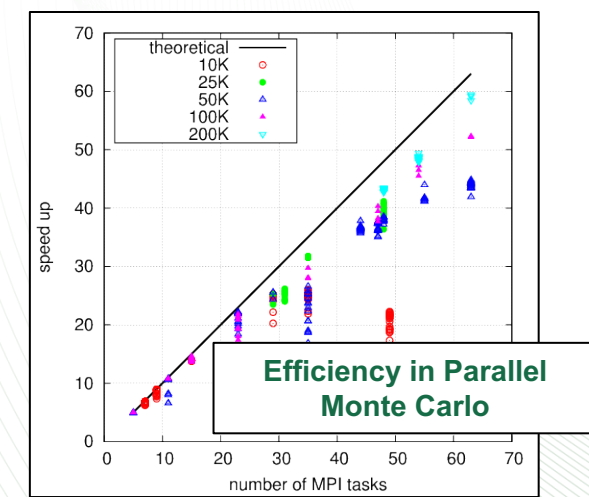
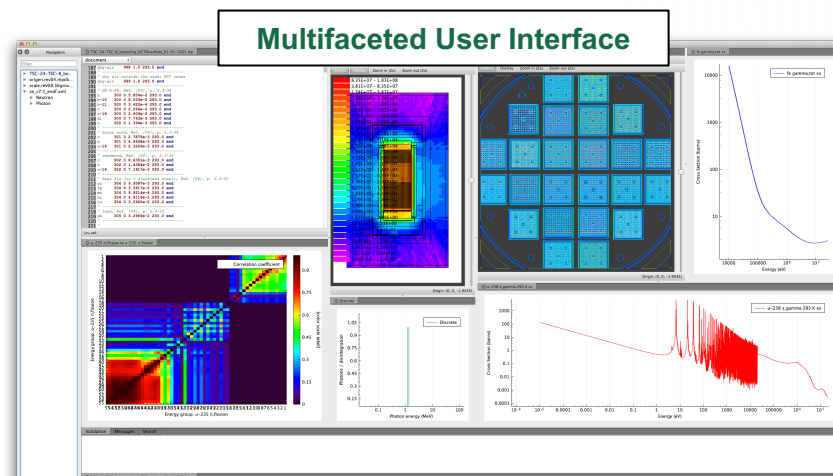
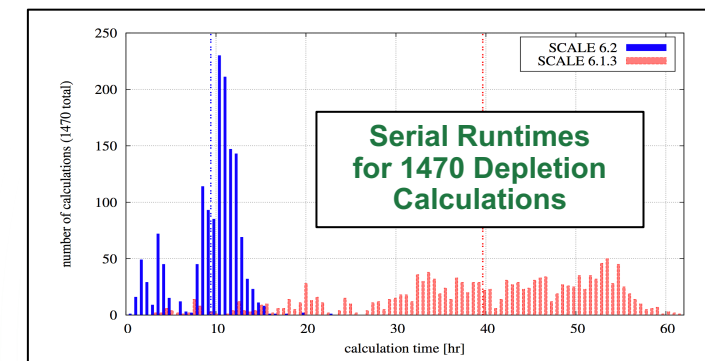
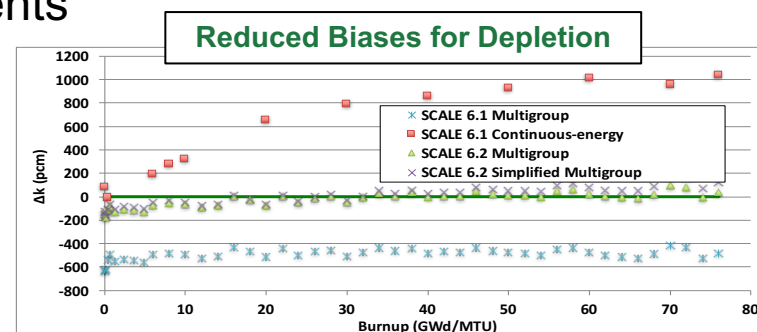
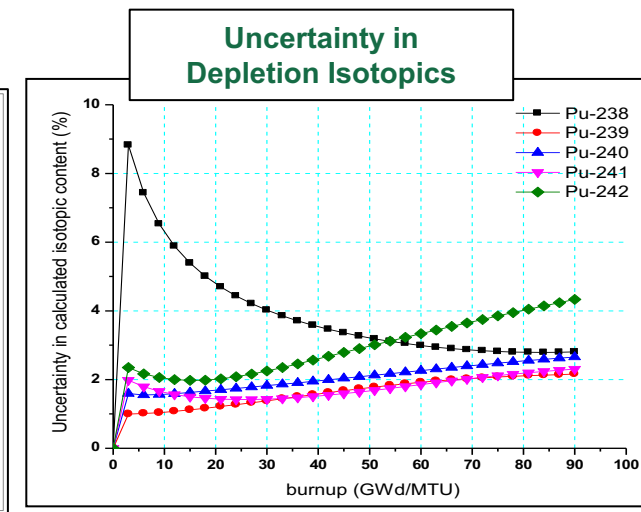
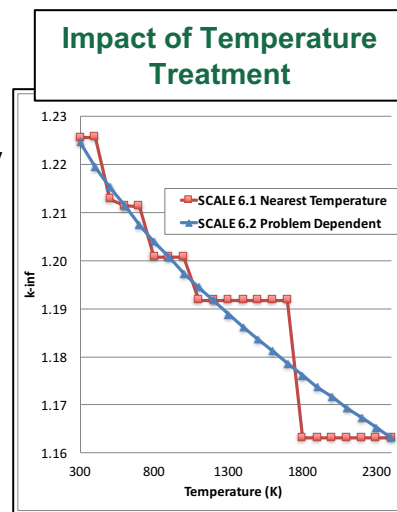
Accurate

- Code and data enhancements to minimize historical biases
- Greatly expanded test suites for validation and verification

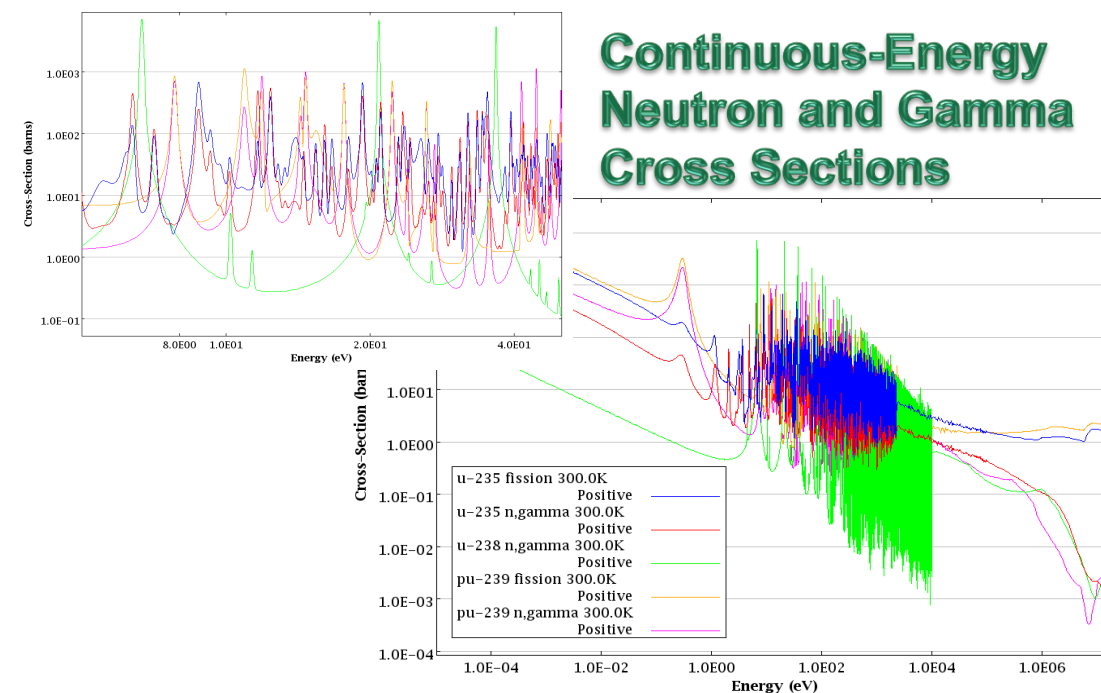
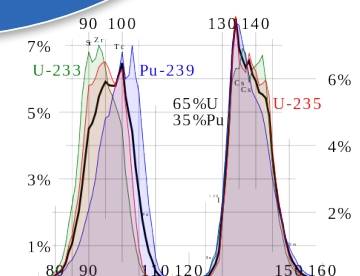
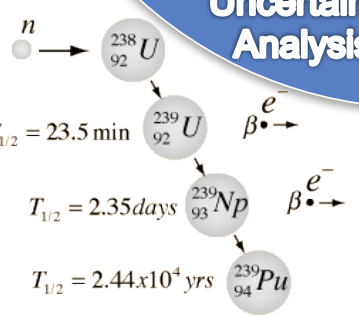
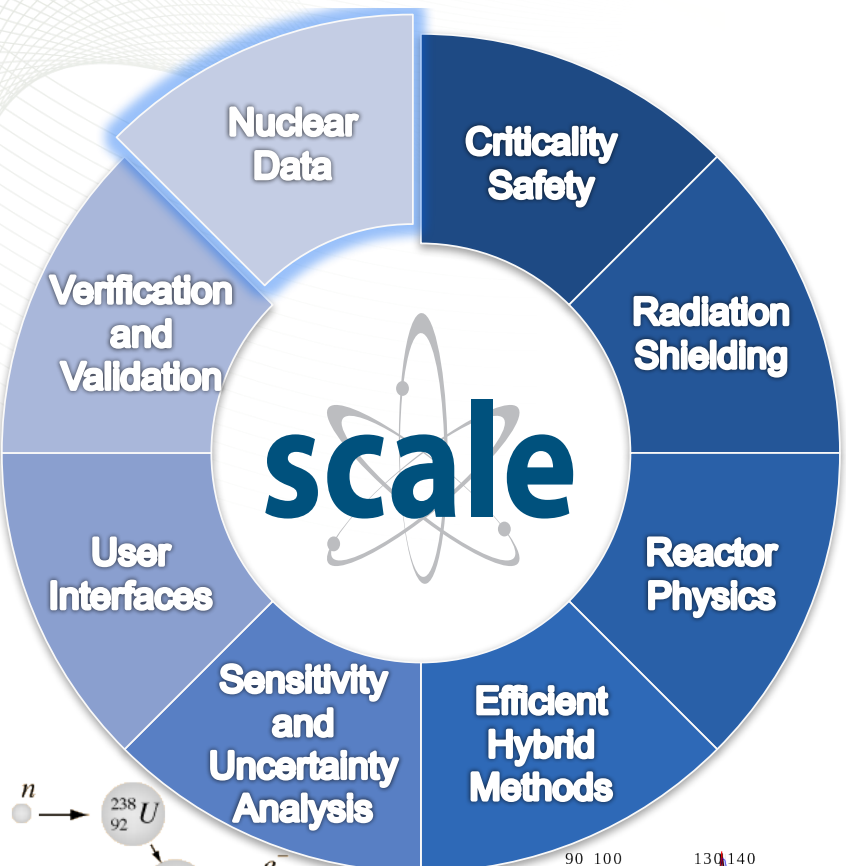
Easy to Use

- Integrated user interface
- Simplified input

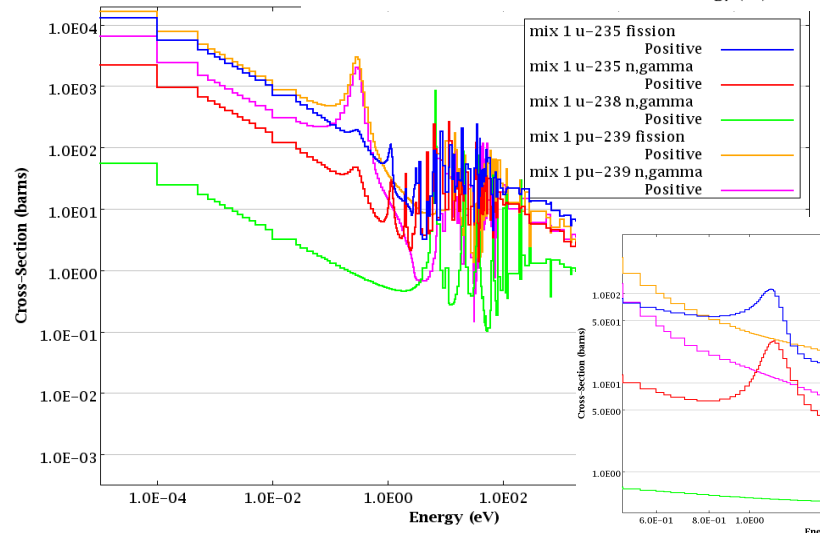
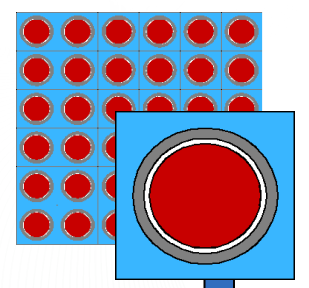
2000 licenses issued through March 2016



# Nuclear Data from ORNL AMPX Tools



### Resonance Self-Shielding



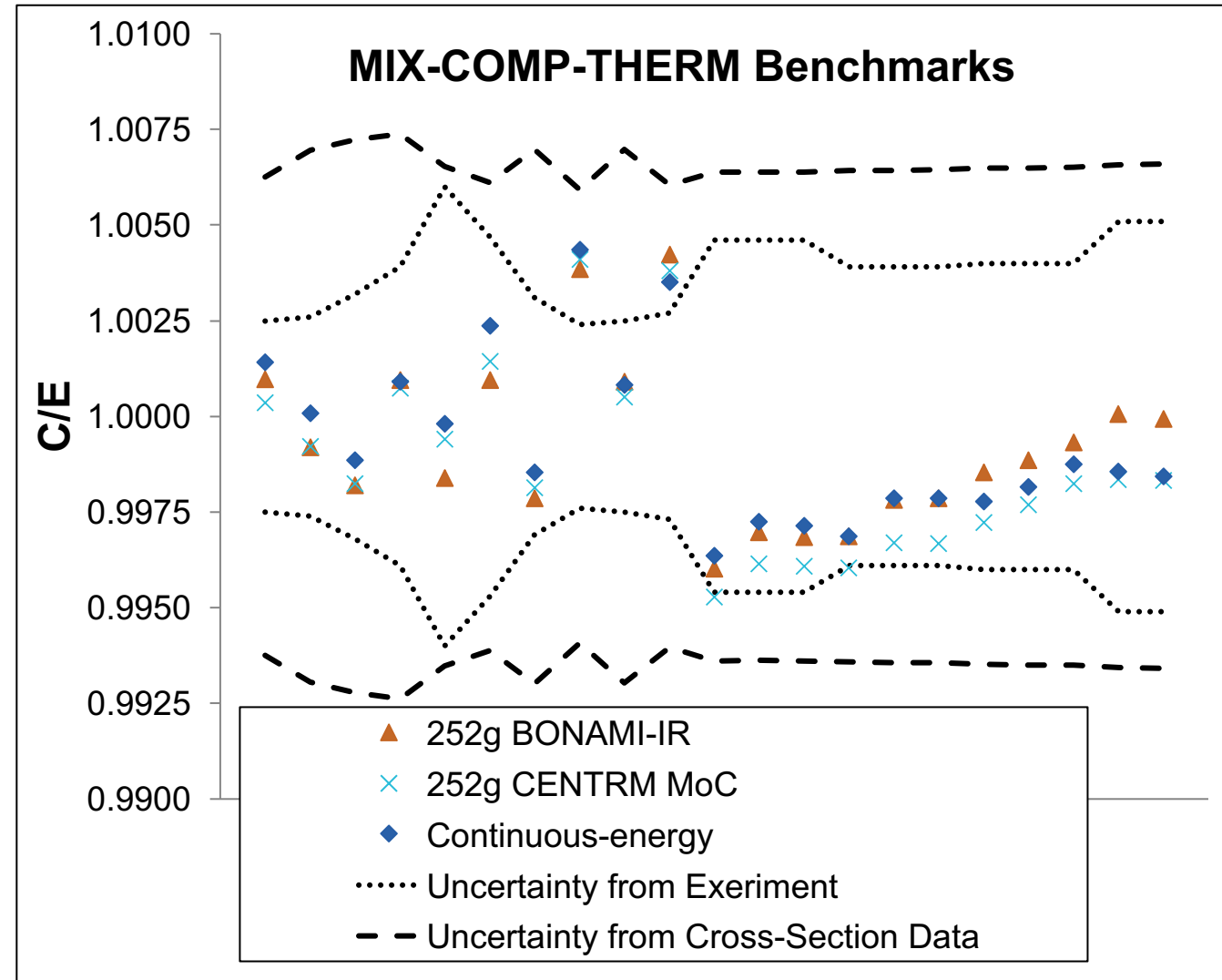
### Multigroup Neutron and Gamma Cross Sections

### 3 Depletion, Decay, and Activation Data

# Cross Section Data

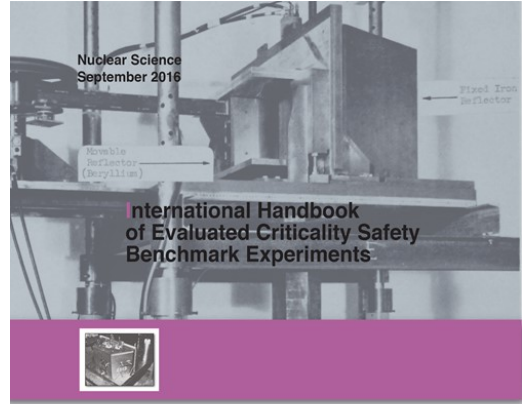
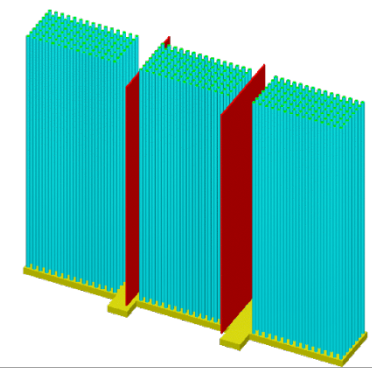
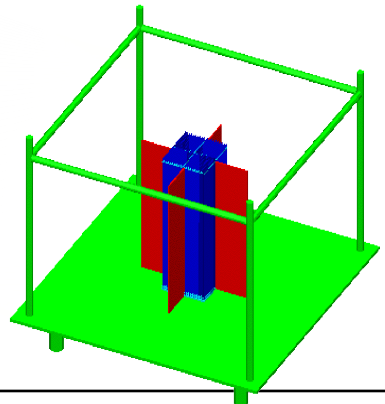
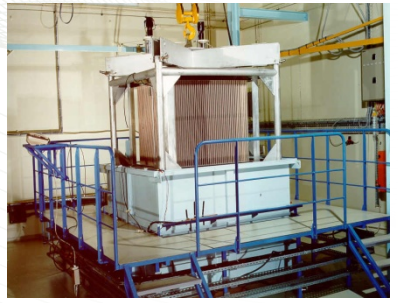
- New CE cross-section data for neutron interactions, [gamma yield](#), and [gamma interactions](#)
- **ENDF/B-VII.1 nuclear data**
- New MG neutron libraries
  - [252-group](#) energy structure
  - [56-group](#) energy structure
  - Intermediate resonance parameters
- Extensive test suite
  - 411 VALID benchmarks
  - 7000 transmission tests
  - 5000 infinite medium tests
- New binary format replacing 40+ year-old AMPX format

AMPX now included with SCALE distribution so users can create their own libraries!



# Validation with critical benchmarks for many types of systems – VALID suite

- 411 configurations from International Criticality Safety Benchmark Evaluation Project (ICSBEP)

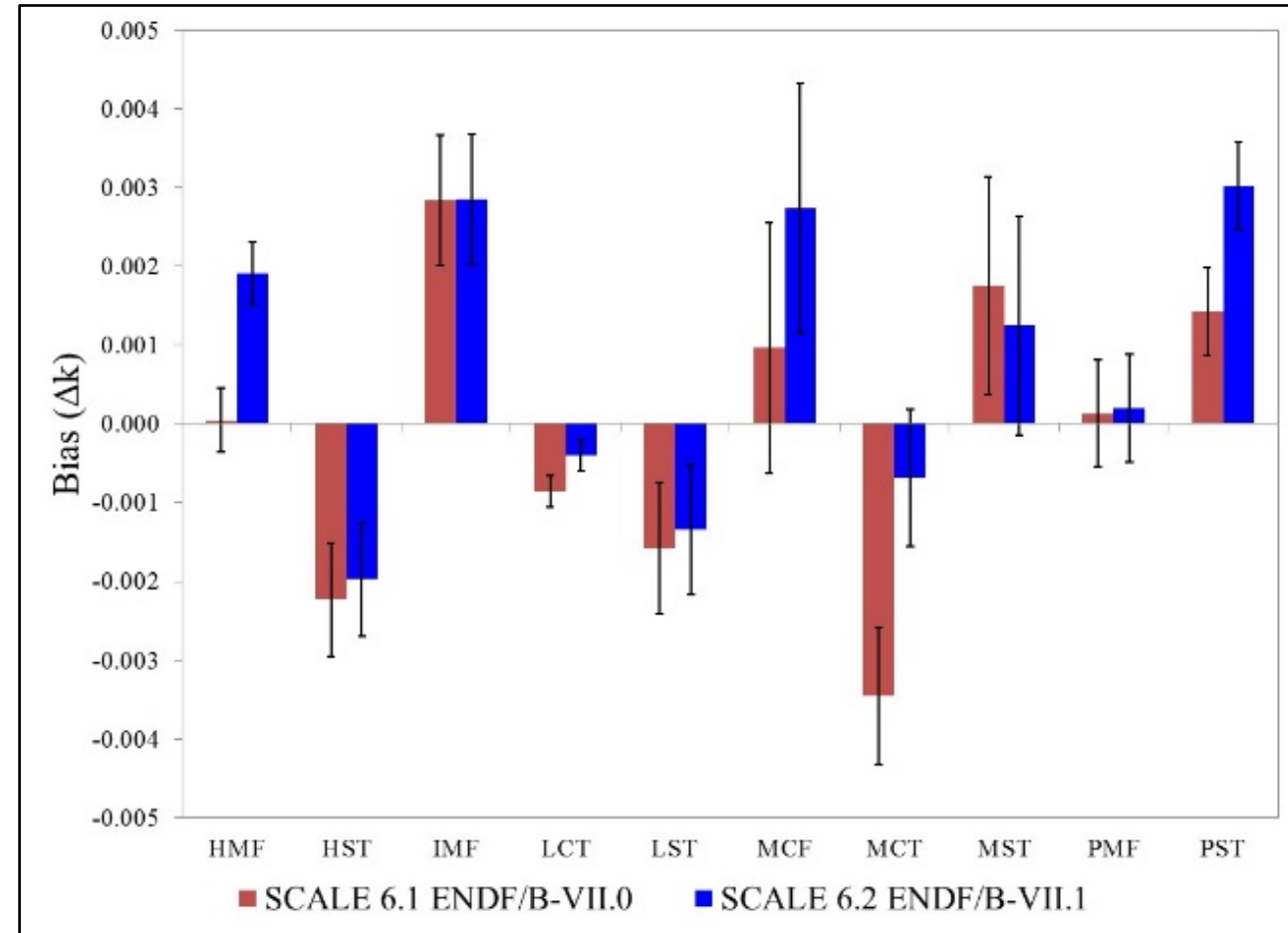
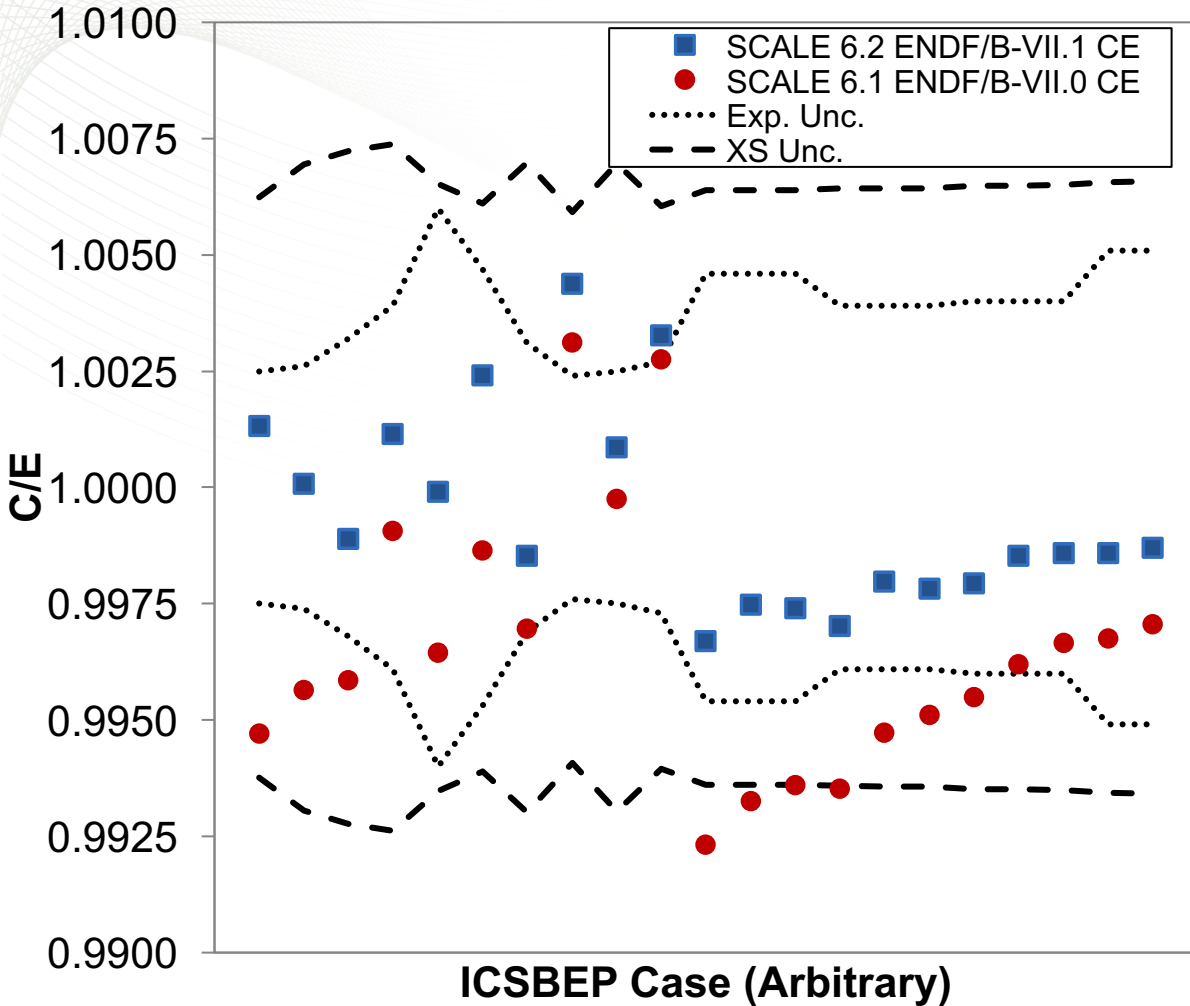


Sequence / Geometry	Experiment class	ICSBEP case numbers	Number of configurations
CSAS5 / KENO V.a	HEU-MET-FAST	15, 16, 17, 18, 19, 20, 21, 25, 30, 38, 40, 65	18
	HEU-SOL-THERM	1, 13, 14, 16, 28, 29, 30	52
	IEU-MET-FAST	2, 3, 4, 5, 6, 7, 8, 9	8
	LEU-COMP-THERM	1, 2, 8, 10, 17, 42, 50, 78, 80	140
	LEU-SOL-THERM	2, 3, 4	19
	MIX-MET-FAST	5, 6	2
	MIX-COMP-THERM	1, 2, 4	21
	MIX-SOL-THERM	2	3
	PU-MET-FAST	1, 2, 5, 6, 8, 10, 18, 22, 23, 24	10
PU-SOL-THERM	1, 2, 3, 4, 5, 6, 7, 11, 20	81	
CSAS6 / KENO-VI	HEU-MET-FAST	5, 8, 9, 10, 11, 13, 24, 80, 86, 92, 93	27
	IEU-MET-FAST	19	2
	MIX-COMP-THERM	8	28

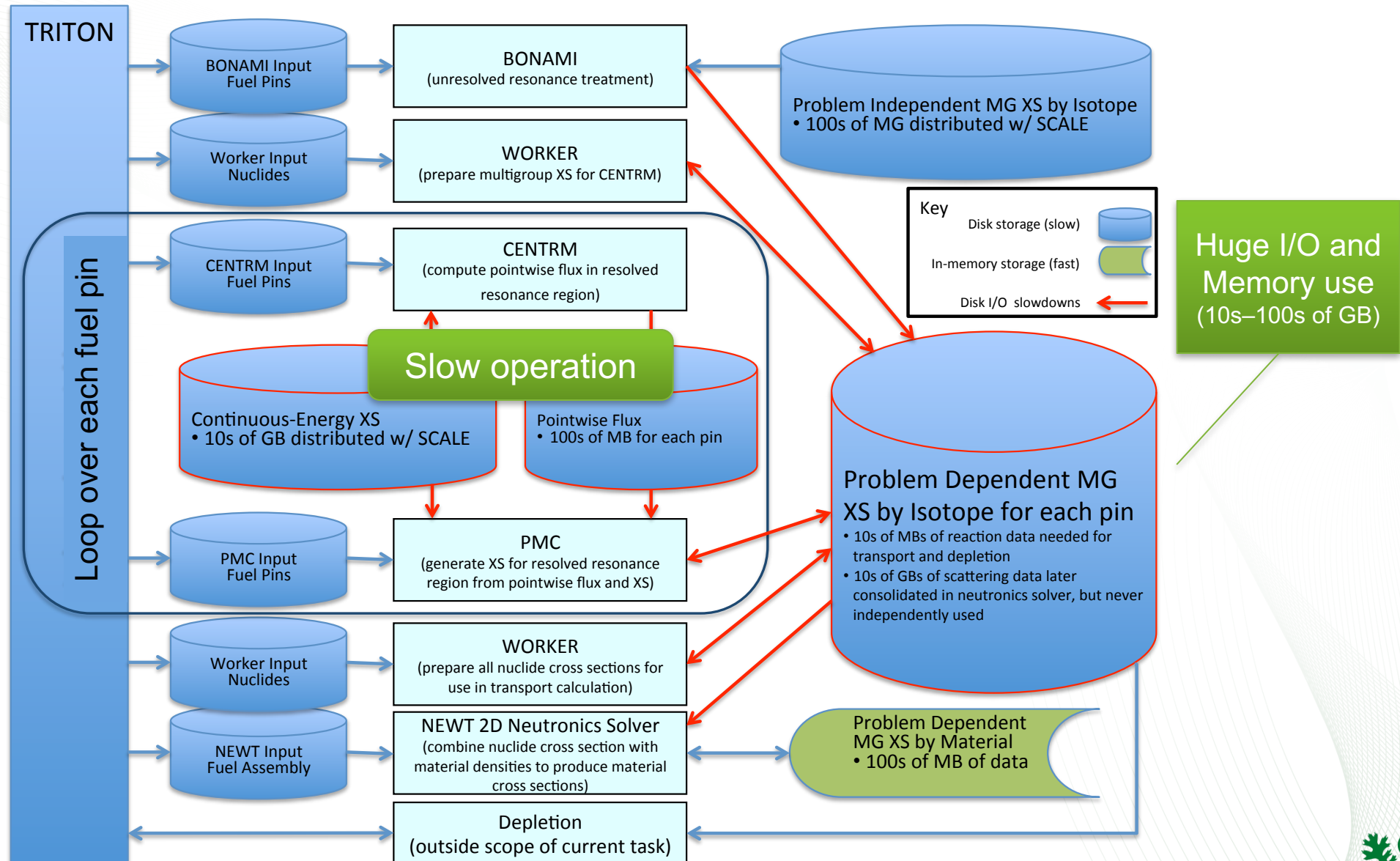
- Fissile materials**
  - High-enriched uranium (HEU),
  - Intermediate-enriched uranium (IEU)
  - Low-enriched uranium (LEU)
  - Plutonium (Pu)
  - Mixed uranium/plutonium oxides (MOX)
- Fuel form**
  - Metal (MET),
  - Fissile solution (SOL)
  - Multi-material composition (e.g. fuel pins – COMP)
- Neutron spectra**
  - Fast
  - Thermal



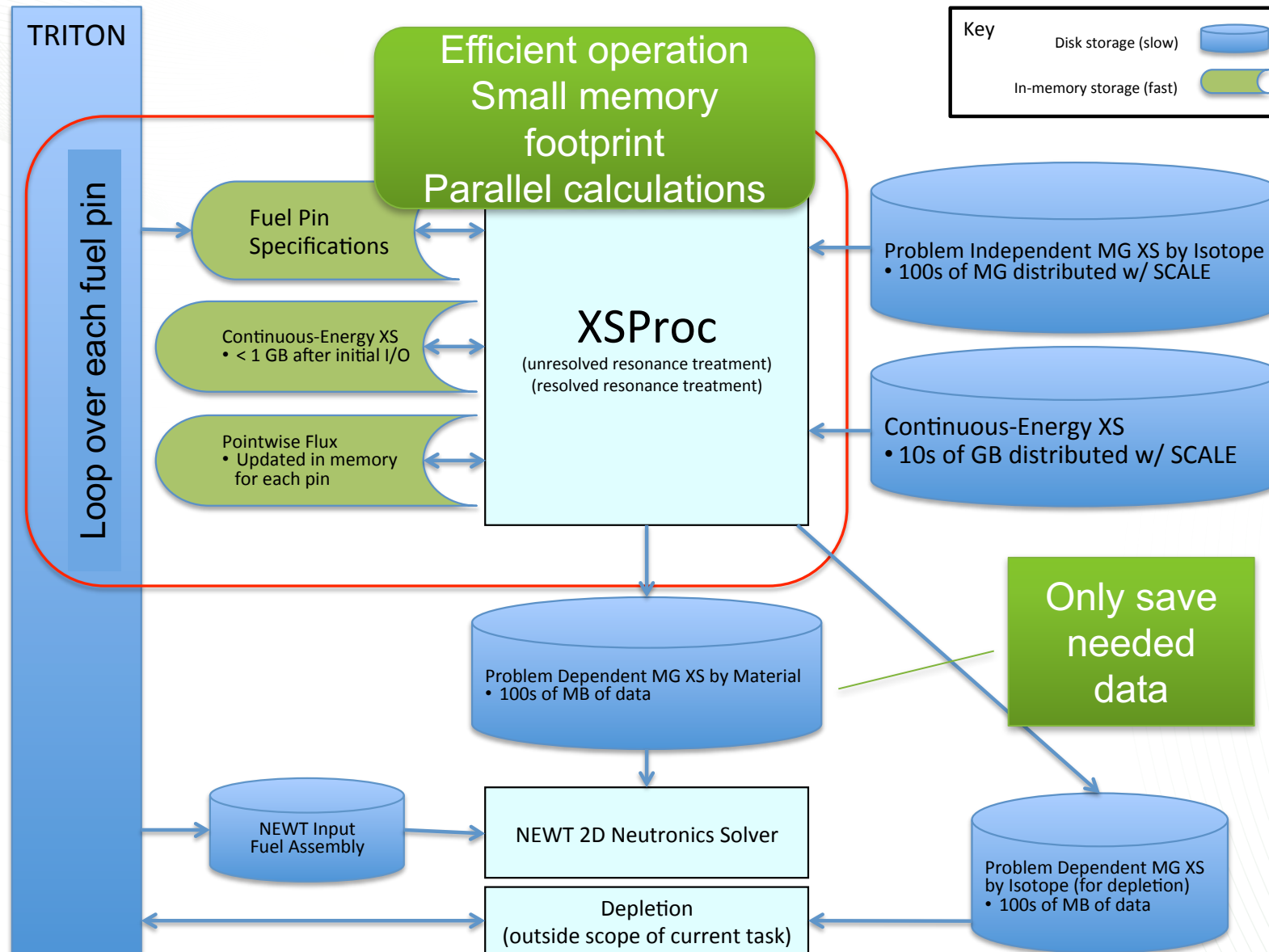
# Comparison SCALE 6.1 - 6.2 results for VALID benchmarks



# SCALE 6.1 Resonance Self-Shielding (somewhat simplified view)



# SCALE 6.2 Resonance Self-Shielding



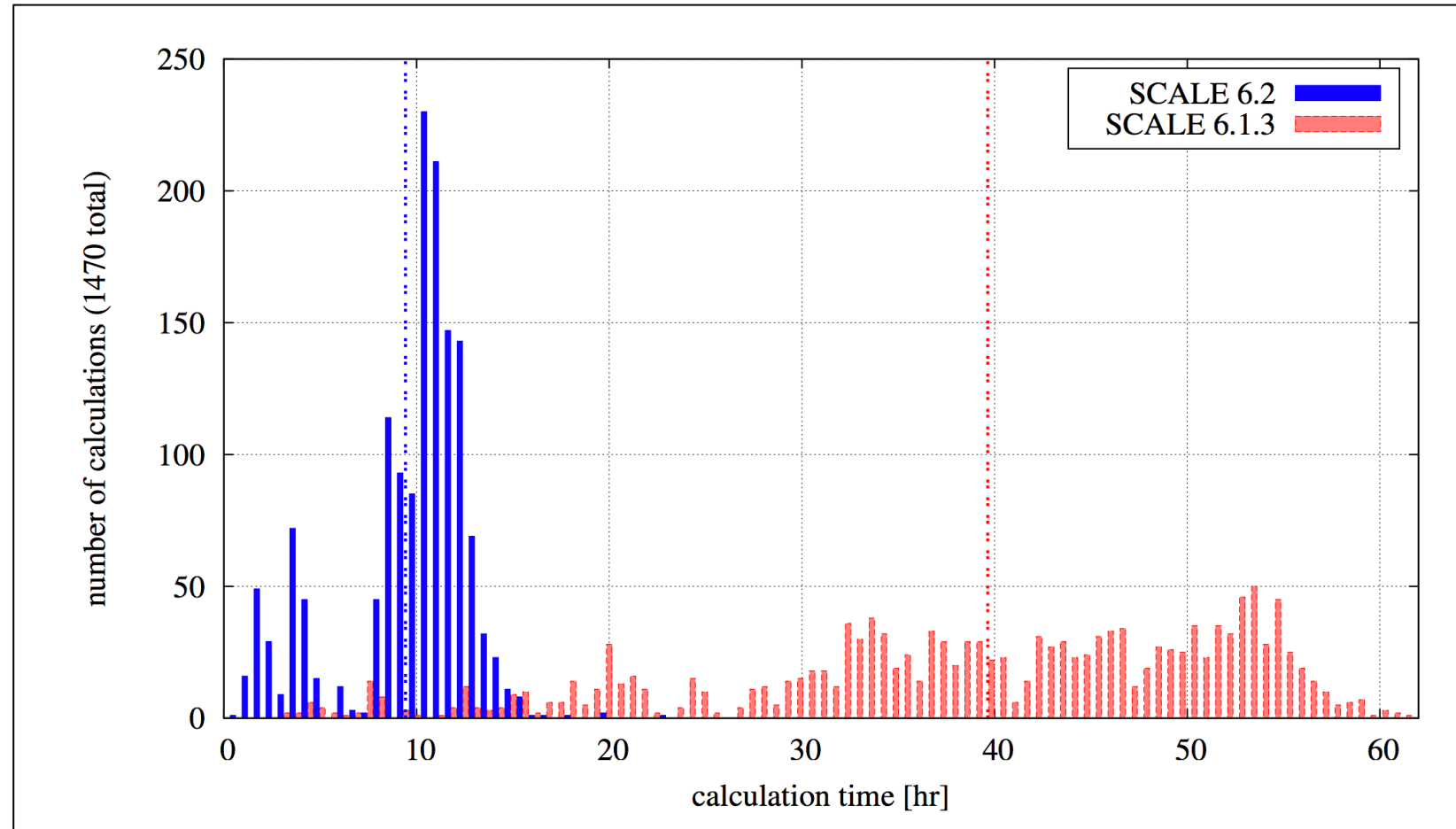


# XSProc features

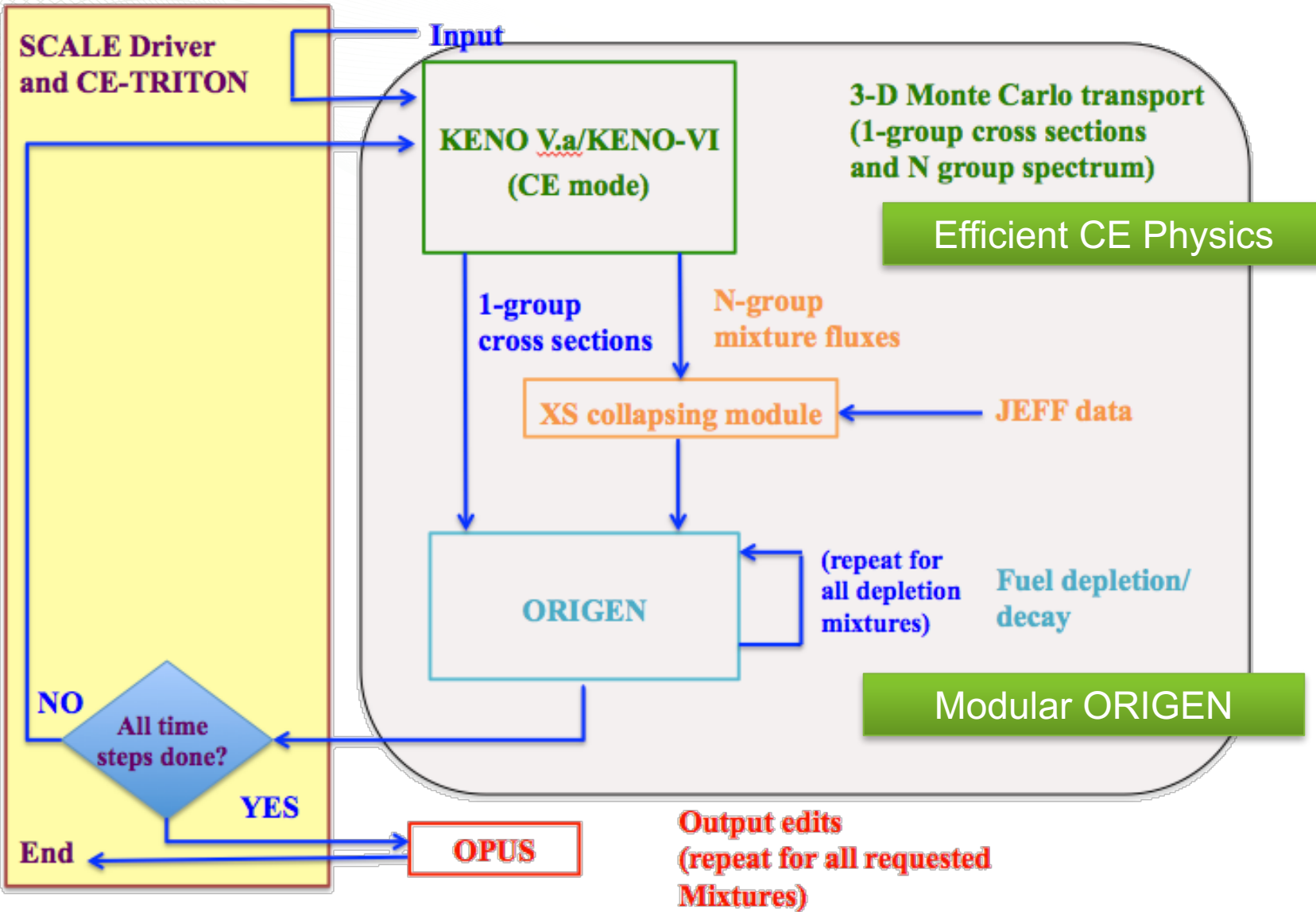
- XSProc provides capabilities for
  - resonance self-shielding of microscopic data,
  - macroscopic cross sections for mixtures,
  - one-dimension MG transport calculations to calculate eigenvalues and flux-weighting functions,
  - group collapsing of cross sections using flux spectra from the one-dimensional eigenvalue calculation or user input fixed source spectra, and
  - spatial homogenization of cross sections across material zones.
- Supported unit cell types
  - repeated lattices without need for Wigner-Seitz approximation
  - double-heterogeneity for HTGR, FHR, and FCM analysis
  - arbitrary slab, cylindrical, or spherical geometry
- Modern C++ architecture with API

# TRITON Runtimes for 1470 ORIGEN Reactor Libraries

Assembly type (Libraries)		Lattice types
PWR	Babcock & Wilcox	15×15
	Westinghouse	14×14, 15×15, 17×17, 17×17-OFA
	Combustion Engineering	14×14, 16×16
	Siemens	14×14, 18×18
BWR	ABB	8×8-1
	Atrium	9×9-9, 10×10-9
	General Electric	8×8-4, 9×9-7, 7×7-0, 8×8-1, 8×8-2, 9×9-2, 10×10-8
	SVEA	64(8×8-1), 96(10×10-4), 100(10×10-0)
MOX	BWR Lattices	8×8-2, 9×9-1, 9×9-9, 10×10-9
	PWR Lattices	14×14, 15×15, 16×16, 17×17, 18×18
	BWR Lattices (×75)	ABB 8×8-1, Atrium 9×9-9, 10×10-9; GE 7×7-0, 8×8-1, 8×8-2, 9×9-2, 10×10-8; SVEA-64, 96, 100
	PWR Lattices (×15)	Siemens 14×14, 18×18; CE 14×14, 16×16; B&W 15×15; Westinghouse 14×14, 15×15, 17×17, 17×17-OFA
Other	AGR (×6)	
	CANDU (×1)	19-pin, 28-pin, 37-pin
	Magnox (×4)	
	RBMK (×24)	
	VVER-440 (×3)	flat, radial enrichments (3.82, 4.25, 4.38)
	VVER-1000 (×7)	flat enrichment



# SCALE 6.2 Depletion with Continuous-Energy KENO

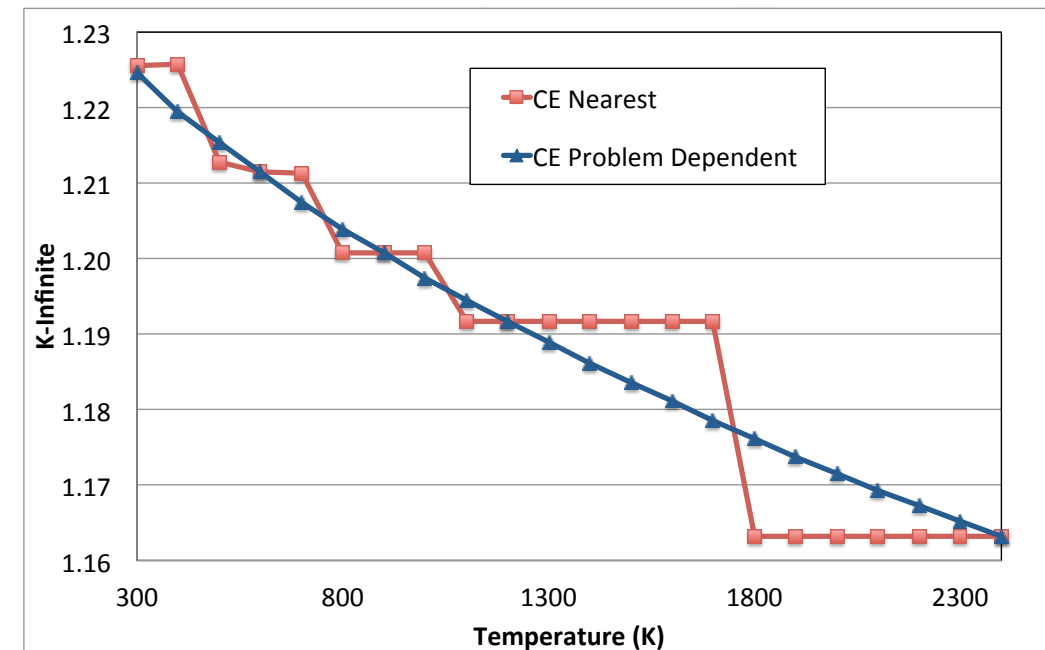
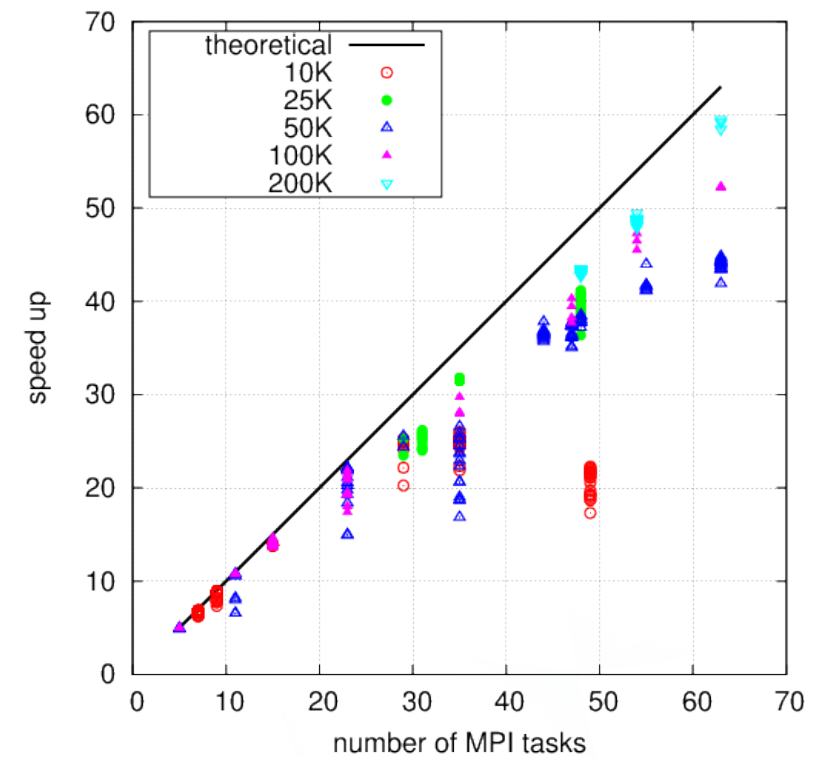


	MG	CE
<sup>234</sup> U	4.03	3.37
<sup>235</sup> U	5.34	5.04
<sup>236</sup> U	1.82	1.65
<sup>238</sup> U	-0.15	-0.15
<sup>238</sup> Pu	-10.38	-10.15
<sup>239</sup> Pu	7.05	5.70
<sup>240</sup> Pu	3.15	2.82
<sup>241</sup> Pu	0.72	-0.05
<sup>242</sup> Pu	-7.40	-6.91
<sup>237</sup> Np	2.33	2.39
<sup>241</sup> Am	-6.56	-7.35
<sup>133</sup> Cs	0.67	0.53
<sup>135</sup> Cs	4.42	4.32
<sup>137</sup> Cs	-2.76	-2.76
<sup>143</sup> Nd	2.09	1.98
<sup>144</sup> Nd	-3.22	-3.15
<sup>145</sup> Nd	-3.01	-3.09
<sup>146</sup> Nd	0.18	0.29
<sup>147</sup> Nd	-0.03	-0.01
<sup>150</sup> Nd	2.58	2.60

**C/E -1 (%) for Spent Fuel Assay Data for a PWR Assembly**

# KENO Improvements

- Substantial **reduction in memory requirements** – over 99% improvement in many cases
- **Accuracy** improvements through comprehensive review and testing
- **Parallel Computations**
  - Significant speedups with MPI on Linux clusters
- Problem-Dependent Doppler broadening for CE calculations for **thermal**, **resolved**, and **unresolved** energy ranges
- Resonance upscatter treatment
  - Significant improvement in elevated temperature CE Monte Carlo
- Source Convergence
  - **Sourcerer** – Hybrid sequence that uses coarse deterministic solution to accelerate fission source convergence
  - Shannon Entropy diagnostics



# Polaris: Fast 2D lattice physics

- **Simple Input**

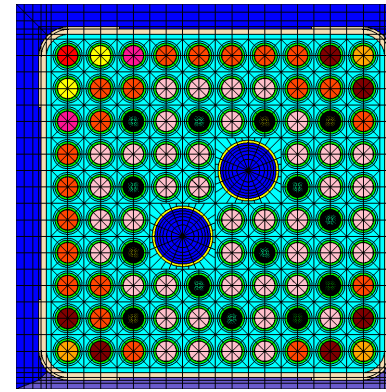
- Assembly geometry
- Material definitions
- Range of system conditions

- **Output**

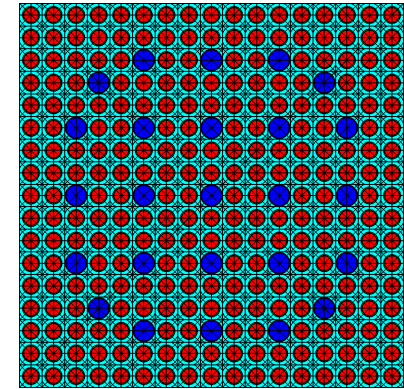
- Lattice physics parameters (.t16 file)
- GENPMAXS converts .t16 file to .PMAX file
- Spent Fuel Isotopics file (.f71 file)

- **Goals**

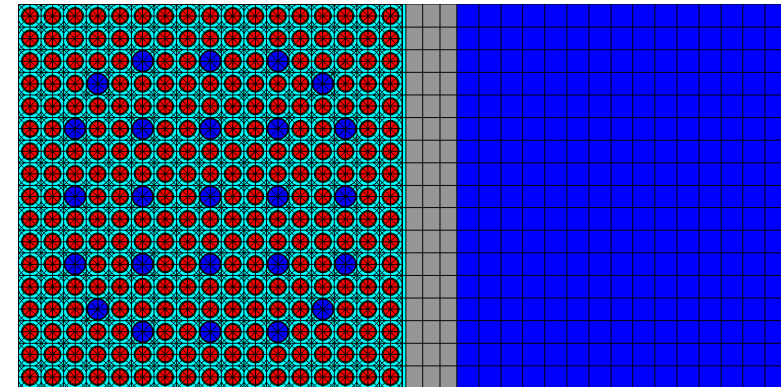
- Fast: < 1 CPU minute per statepoint
- Simple Input: 100 - 200 lines
- Target accuracy compared to Monte Carlo:
  - 200 pcm dk
  - BWR: 1% RMS, 1.5% Max pin fission rate error
  - PWR: 0.5% RMS, 1.0% Max pin fission rate error
- Good agreement with radiochemical assay data



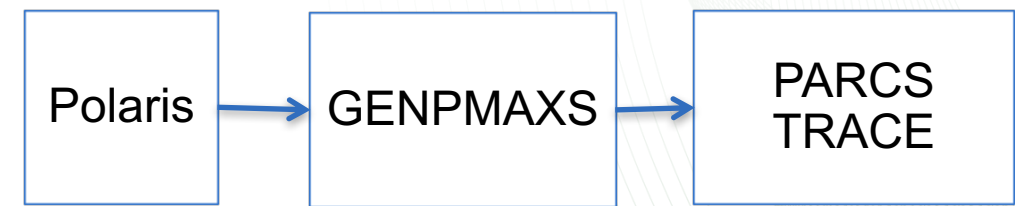
BWR GE 10x10



PWR W 17x17

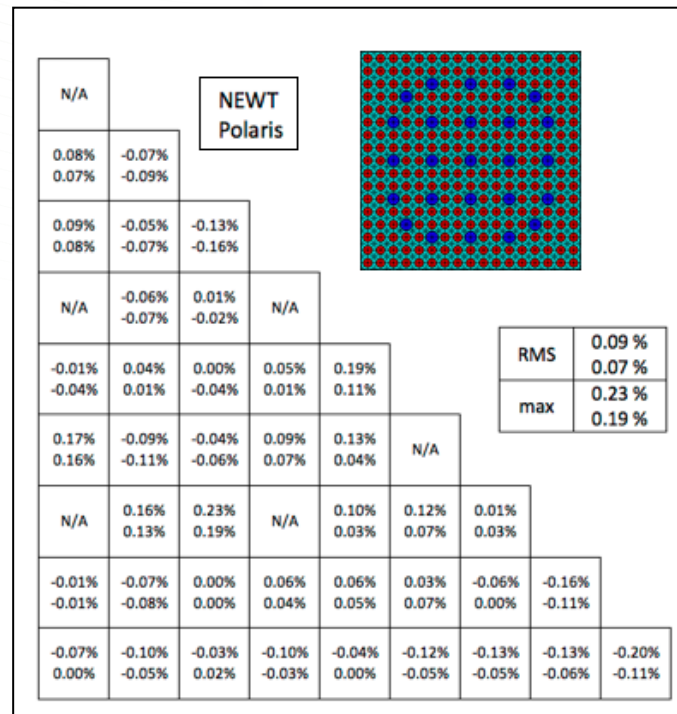


Reflector Model

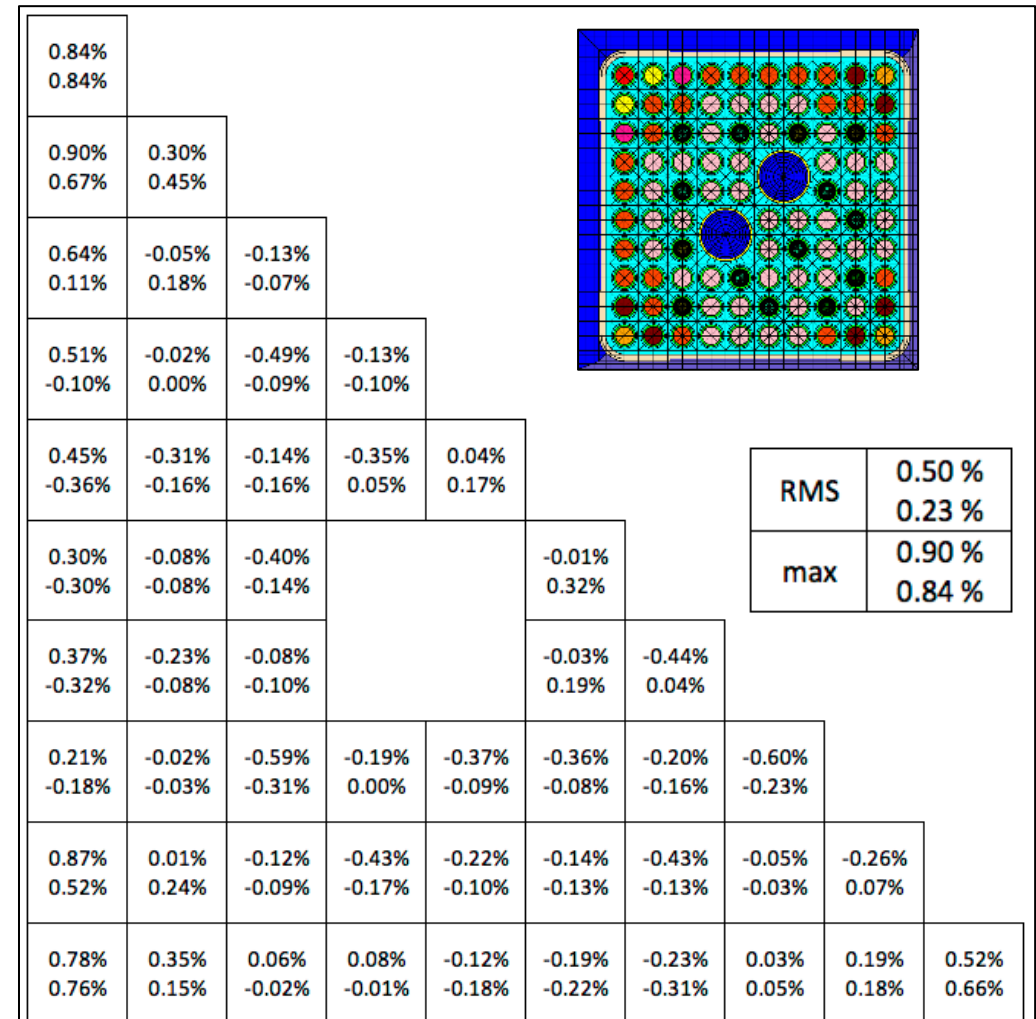


# Pin power prediction

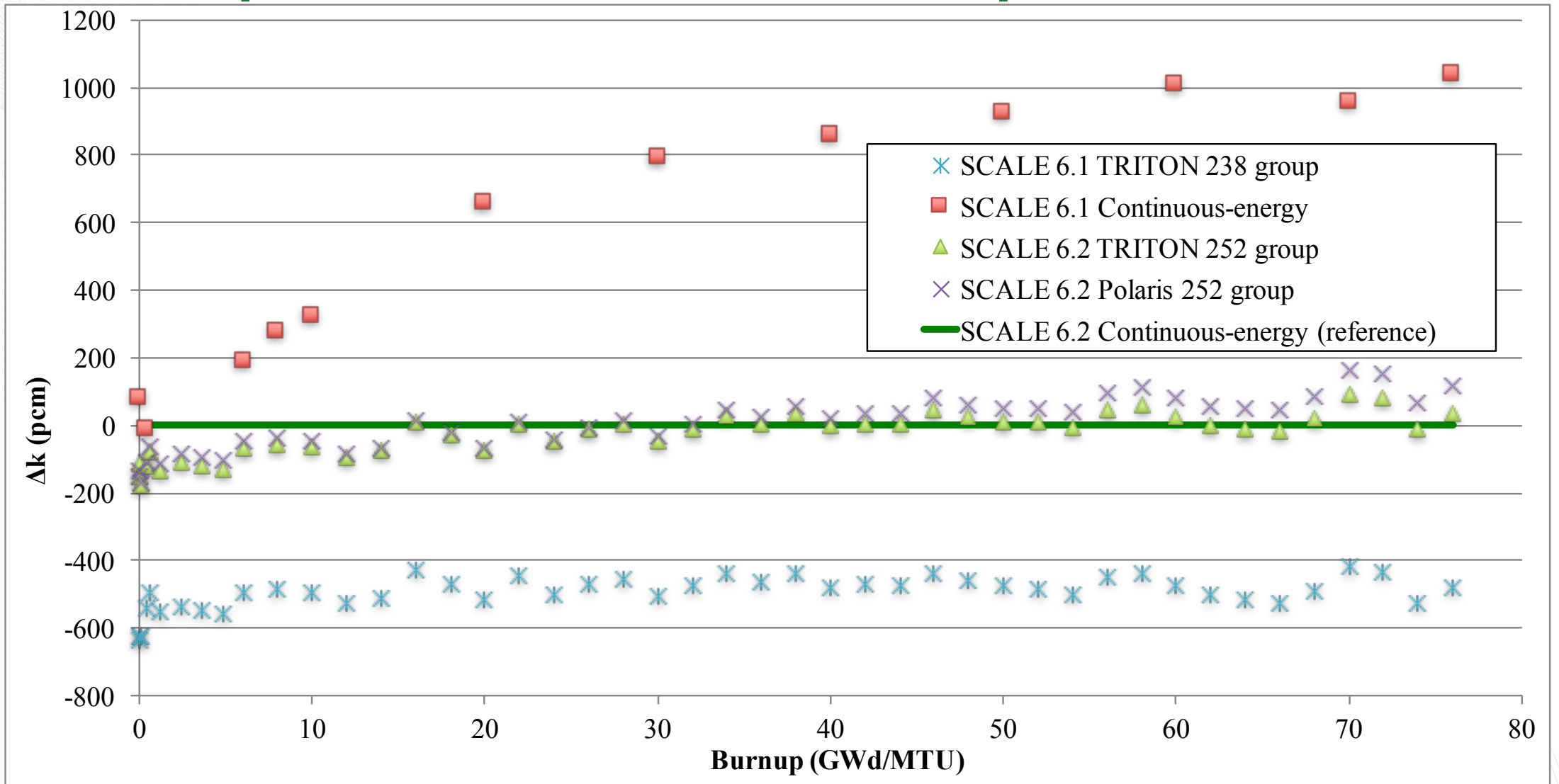
17×17 PWR lattice pin fission rate differences at nominal conditions for TRITON and Polaris



10×10 BWR lattice dominant zone pin power differences at 40% void fraction for TRITON and Polaris



# Eigenvalue prediction for PWR fuel depletion



# Conclusions

- SCALE 6.2 provides many improvements in accuracy and runtime, especially for reactor physics calculations.
- CE Monte Carlo biases for MOX or burned-fuel calculations have been minimized with numerous revisions to the CE data and physics implementation in the codes.
- Monte Carlo innovations enable CE depletion with parallel Monte Carlo and integrated problem-dependent Doppler broadening.
- Historical biases in MG LWR calculations have reduced to approximately 100 pcm through many improvements in the nuclear data, group structures, and resonance self-shielding techniques.
- Runtimes for lattice physics calculations are greatly improved with the availability of the new Polaris tool as well as numerous enhancements in XSPROC and NEWT as applied in TRITON.
- Custom nuclear data libraries can now be created with the inclusion of AMPX.